September 23, 1985

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Docket No. 50-320
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Docket No. 50-320

Mr. F. R. Standerfer
Vice President/Director
Three Mile Island Unit 2
GPU Nuclear Corporation
P.O. Box 480
Middletown, PA 17057

Dear Mr. Standerfer:

Subject: Three Mile Island Unit 2
License No. DPR-73
Docket No. 50-320
Corrections to Recovery Operations Plan
Change Approval No. 29 (Request No. 46)

On August 8, 1985, the staff issued an Amendment of Order with a supporting safety evaluation (SE) approving the modifications to the Proposed Technical Specifications (PTS) designated as Technical Specification Change Request No. 46. The associated Recovery Operations Plan (ROP) Change Request No. 46 was also approved on August 8, 1985, and is hereby designated as ROP Change Approval No. 29. This approval was also issued with a supporting SE, which was based largely on the SE included with the Amendment of Order.

The purpose of this letter is to correct certain editorial errors associated with ROP Change Approval No. 29. The enclosed corrections do not affect the conclusions of the previously issued SE's. A discussion of each correction follows:

1) Page 4.1-2: Deleted. Based on the changes to Section 4.1.1 of the ROP included in Approval No. 29, page 4.1-1 was correctly modified. However, the requirements of that section were also included on page 4.1-2, which was not changed. The enclosed correction deletes the requirements for boron injection previously listed on page 4.1-2, consistent with ROP Change Approval No. 29.

2) Table 4.3-3: When Table 4.3-3 was modified, certain radiation monitoring requirements were inadvertently dropped from the table. These requirements, 1.b., Reactor Building Equipment Doors (AMS-3), and 1.d., CACE Vent Monitor, have been returned to Table 4.3-3 in the enclosed corrected page 4.3-4. Additional notes to the table have also been restored on corrected Page 4.3-5.

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DATE	9/20/85	9/24/85	9/23/85				

3) Table 4.3-7: When Table 4.3-7 was modified, certain monitoring instrumentation was inadvertently dropped from the table. Item 11, Spent Fuel Storage Pool "A" Water Level and item 12, Fuel Transfer Canal (deep end) Water Level have been returned to Table 4.3-7 in the enclosed corrected Page 4.3-10. The notes for this table appear on Page 4.3-10a. Page 4.3-11 should be deleted, as Table 4.3-7 appears on Page 4.3-10.

4) Page 4.4-1: Section 4.4.3, SAFETY VALVES, should read "deleted" instead of "not applicable." The enclosed corrected Page 4.4-1 restores the term "deleted," in accordance with ROP Change No. 22.

The above changes have been discussed with your staff and will become effective September 23, 1985, the effective date of ROP Change Approval No. 29. If you have any questions regarding these corrections, please contact Mr. Randy Hall at (301) 492-8095.

Sincerely,

Original signed by
B. J. Snyder

Bernard J. Snyder, Program Director
Three Mile Island Program Office
Office of Nuclear Reactor Regulation

Enclosure:
Corrected Pages for ROP Change
Approval No. 29

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Enclosure

FACILITY OPERATING LICENSE NO. DPR-73

DOCKET NO. 50-320

Replace the following pages of the TMI-2 Recovery Operations Plan with the enclosed pages as indicated:

4.1-2

4.3-4

4.3-5

4.3-10

4.3-11

4.4-1

This page deleted.

TABLE 4.3-3

RADIATION MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

FUNCTIONAL UNIT	CHANNEL CHECK	CHANNEL CALIBRATION	CHANNEL FUNCTIONAL TEST	MINIMUM CHANNELS OPERABLE	APPLICABILITY	ACTION
1. CONTAINMENT						
a. Reactor Building Purge Sampler (AMS-3)	D	SA	W	1	Note 1	Note 3
b. Reactor Building Equipment Doors (AMS-3)	D	SA	W	1	Note 10	Note 7
c. Deleted						
d. CACE Vent Monitor	D	SA	W	1	Note 1	Note 9
2. FUEL HANDLING BUILDING EXHAUST MONITORS (HPR-221A or HPR-221B)						
a. Gaseous Activity	S	R	M	1	Note 5	Note 6
b. Particulate Activity	S	R	M	1	Note 5	Note 6
3. SDS MONITORS						
a. Process Monitor (IX04)	S	R	M	1	Note 1	Note 4
b. Area Monitor (IX03)	S	SA	M	1	Note 1	Note 2
4. Deleted						

(See following pages for notes)

TABLE 4.3-3 (Con't)

RADIATION MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

NOTES:

1) During operation of the monitored system.
2) If monitor becomes inoperable, repair or replace by equivalent equipment within 12 hours. If not completed within 12 hours terminate operation of the monitored system and restore the inoperable monitor(s) to operable status.
3) Restore the inoperable monitor(s) to operable status within 72 hours.
4) If ion exchange effluent monitor is inoperable, sample on 4 hour frequency for gross beta. If inoperable longer than 24 hours then terminate operation of ion exchange system and restore the inoperable monitor to operable status.
5) With radioactive waste in the fuel handling building.
6) With the required instrumentation inoperable, suspend all operations involving movement of radioactive wastes in the fuel handling building, restore the inoperable equipment to OPERABLE status within 48 hours.
7) With the AMS-3 inoperable, close at least one of the Reactor Building Equipment Doors and restore the inoperable equipment to operable status prior to the reopening of both Equipment Doors.
8) Deleted
9) Two filter trains and associated monitors are normally available. If one monitor becomes inoperable, discontinue operation through the effected filter train and transfer operations by the operable filter train. If both monitors become inoperable, repair or replace by equivalent equipment within twelve (12) hours or conduct air sampling using alternate methods on a four (4) hour frequency. If repair or replacement is not completed within seven (7) days, terminate operation of the system until at least one monitor is returned to operable status.
10) With both reactor building equipment hatch airlock doors open simultaneously.

TABLE 4.3-7

ESSENTIAL PARAMETERS MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

INSTRUMENT	CHANNEL CHECK	CHANNEL[1] CALIBRATION	READOUT LOCATION(S)	MINIMUM OPERABLE CHANNELS
1. Reactor Building Pressure	S	R	Control Room	2
2. Reactor Vessel Water Level	S/W[2]	SA	Control Room [2]	2[2]
3. Deleted				
4. Incore Thermocouples	S	R	Control Room or Cable Room	2
5. NI Intermediate Range Level Log N	M	R	Cab 217 & Control Room	1
6. NI Source Range Level	M	R	Cab 217[4] & Control Room	2
7. Reactor Building Water Level	NA	SA	Control Bldg. Area West	1
8. Borated Water Storage Tank Level	S	R	Control Room	1
9. Steam Generator Level	NA	NA	NA	1/Generator
10. Deleted.				
11. Spent Fuel Storage Pool "A" Water Level	S/W[2]	SA	Control Room [2]	2[2]
12. Fuel Transfer Canal (deep end) Water Level	S/W[2]	SA	Control Room [2]	2[2]

(See following page for notes)

THIS PAGE DELETED

4.4 REACTOR COOLANT SYSTEM

REACTOR COOLANT LOOPS

4.4.1 Verify that surveillance of the Reactor Coolant System is being performed in accordance with procedures approved pursuant to Technical Specification 6.8.2.

4.4.2 REACTOR VESSEL WATER LEVEL MONITORING

4.4.2 The Reactor Vessel Water Level Monitoring Instrumentation shall be demonstrated OPERABLE as required by Table 4.3-7.

SAFETY VALVES

4.4.3 Deleted.

4.4.9 PRESSURE/TEMPERATURE LIMITS

REACTOR COOLANT SYSTEM

4.4.9.1.1 Deleted.

4.4.9.1.2 Deleted.

4.4.9.1.3 Deleted.

4.4.9.1.4 The pH of the reactor coolant shall be determined to be greater than or equal to 7.5 and less than 8.4 at least once per 7 days.

4.4.9.1.5 The chloride concentration in the reactor coolant shall be determined to be less than or equal to 5 ppm at least once per 7 days.